Uranium silicides have proven of interest as advanced technology reactor fuels due to their enhanced thermal conductivity and high uranium density ($\text{U}_3\text{Si}$ and $\text{U}_3\text{Si}_2$) compared to traditional $\text{UO}_2$. However, susceptibility to oxidation and wash out, in the event of a cladding breech, could limit the potential for deployment of silicides as accident tolerant fuels. Mitigating the water reaction for $\text{U}_3\text{Si}_2$ could enable its use as an accident tolerant, high uranium density fuel or as a composite fuel constituent. Presented will be the steam oxidation behavior of $\text{U}_3\text{Si}_2$ alloyed with $\text{Al}$, $\text{Cr}$, $\text{Y}$, $\text{Nb}$, and $\text{Zr}$ ranging from 2-12 volume percent alongside screening data for unalloyed $\text{U}_3\text{Si}_2$ and $\text{UO}_2$. It has been identified that at alloying levels above 6vol%, the steam oxidation dynamics are altered, from non-alloyed $\text{U}_3\text{Si}_2$, under thermally ramped conditions. The modified reaction kinetics for the alloyed compositions will be presented and discussed. Additionally, the microstructural degradation of the alloyed compositions and x-ray power diffraction patterns of the oxidation products will be presented.