

Radiation effects on mechanical behaviour of zirconium alloys: from dislocations to polycrystals

F. Onimus¹, L. Dupuy¹, M. Bono², F. Momprou³, P. Pilvin⁴

¹ DEN - Service de Recherches de Métallurgie Appliquées, CEA Saclay, Université Paris-Saclay, 91191 Gif-sur-Yvette, France

² DEN - Service d'Etude des Matériaux Irradiés, CEA, Université Paris-Saclay, 91191 Gif-sur-Yvette, France

³ Centre d'Elaboration de Matériaux et d'Etudes Structurales, CNRS, 29 Rue Jeanne Marvig, 31055 Toulouse, France

⁴ Institut de Recherche Dupuy de Lôme, Université de Bretagne Sud, 27 rue Armand Guillemot, 56321 Lorient, France

Zirconium alloys are used as cladding tubes for the fuel of nuclear water reactors. In-reactor, they are subjected simultaneously to mechanical loadings and neutron irradiation. In order to gain a better understanding of radiation effects on the mechanical behavior, deformation mechanisms have been studied starting from the smallest length scale up to the macroscopic mechanical behavior. Mechanical tests have been conducted on neutron-irradiated tubes. The deformation mechanisms have been analyzed by TEM after testing but also on Zr-ion irradiated samples thanks to in situ straining inside the TEM. A process of removal of radiation defects by gliding dislocations has been observed. Interactions between dislocations and radiation defects have been studied by molecular and dislocation dynamics. This numerical study has been able to provide a good understanding of the clearing of radiation defects depending on the gliding plane of dislocations. Finally, a polycrystalline model, that takes into account these mechanisms, has been developed and compared to mechanical tests with complex loading path. A cyclic strain softening, due to the removal of radiation defects, is predicted by the model in good agreement with experiments.